

Extension of Operational Regime in High-Temperature Plasmas and Effect of ECRH on Ion Thermal Transport in the LHD

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Abstract. Simultaneous high ion temperature (T_i) and high electron temperature (T_e) regime was successfully extended due to the optimized heating scenario in the LHD. Such high-temperature plasmas were realized by the simultaneous formation of an electron internal transport barrier (ITB) and an ion ITB by the combination of high power NBI and ECRH. Although the ion thermal confinement was degraded in the plasma core with increase of T_e/T_i by the on-axis ECRH, it was found that the ion thermal confinement was improved at the plasma edge. The normalized ion thermal diffusivity $\chi_i/T_i^{1.5}$ at the plasma edge was reduced by 70%. The improvement of the ion thermal confinement at the edge led to increase in T_i in the entire plasma region even though the core transport was degraded.

1. Introduction

Confinement improvement is necessary for realization of high-temperature plasmas and is one of the most important issues in toroidal devices. Since the first observation of H mode [1], various kinds of improved confinement modes have been observed in tokamaks and helical/stellarator devices. Of these improved confinement modes, an internal-transport barrier (ITB) is characterized by a steep gradient formation in temperature profiles and a decrease in thermal diffusivity in a plasma core region [2-4].

In future reactors, the fusion reaction is expected to be sustained under the electron heating dominant condition, where both the ion temperature (T_i) and the electron temperature (T_e) are high. Thus not only the investigation of the confinement improvement but also the characterization of the thermal transport for the plasmas, of which T_i and T_e are simultaneously high, are necessary. In the present status, such a plasma condition is realized by the combination heating of a neutral beam injection (NBI) and an electron cyclotron

resonance heating (ECRH). The effect of a T_e/T_i and/or an ECRH on the ion thermal confinement has been studied in several devices [5-9].

In the Large Helical Device (LHD), electron-ITB (e-ITB) relating to the production of high T_e plasmas has been realized by strongly centre-focused ECRH beams [10-13]. The radial electric field E_r was measured by charge exchange recombination spectroscopy (CXRS) [14] and/or the heavy ion beam probe (HIBP) [15]. The formation of the strong positive E_r was observed in the core region [10], which well agreed with the prediction of the neoclassical transport theory, and the electron thermal confinement improved by a factor of 8 inside the ITB. An e-ITB have been observed in other helical devices as summarized in Ref. [16] and the interest has renewed since beginning of an operation of the Wendelstein 7-X, with Core Electron Root Plasmas [17]. High- T_i accompanied with an ion-ITB (i-ITB) has been achieved in NBI-sustained plasmas in combination with a carbon pellet [18-22]. The gradient of T_i at the core region increased and the ion thermal diffusivity was clearly reduced. The i-ITB observed in the LHD is characterized by centre-peaked T_i profile in contrast with a so-called box-type ITB in tokamaks [2, 23]. The momentum transport was found to be improved and the strong-peaked profiles of the toroidal flow velocity were formed [21, 22]. The formation of weak negative E_r in the core and the strong positive E_r in the peripheral region were also found [21].

In the previous experiments above, an i-ITB and an e-ITB have been obtained independently [22], indicating decoupling with each other, which is different from tokamak ITBs. Therefore i-ITB and e-ITB had been studied individually. In recent years, an integration of high T_i and high T_e with the simultaneous formation of an i-ITB and an e-ITB has been successfully achieved in the LHD by the combination of the NBI and the ECRH [8].

The present paper shows the high-temperature plasma regime extended recently and the effect of an ECRH on the ion thermal transport in the LHD and is organized as follows. In section 2, the experimental setup is described. Section 3 treats the extension of the operational regime in high temperature plasmas in the LHD. The temperature profiles are also compared among the four different confinement states. Section 4 focuses on the effect of the ECRH on T_i profile. The response of the T_i gradients to the ECRH and the dependence of the change in the ion temperature profile during the ECRH on the electron density (n_e) are described. Section 5 is discussion. The effect of the ECRH on the the ion thermal diffusivity and the response of the density fluctuation to the ECRH are shown. The summary is in section 6.

2. Experimental Setup

The LHD is the largest heliotron device with a major radius $R = 3.9$ m, a minor radius $a = 0.6$ m, a pole number of 2 and a toroidal period of 10 [24, 25]. The heliotron configurations are produced by a set of helical winding coils and three sets of poloidal field coils, which are all superconducting magnets. The magnetic field on the axis is ~ 2.9 T. In the LHD, three negative-ion-based NBIs produce hydrogen neutral beams with the beam energy of 180 keV and total port-through power of 16 MW [26]. The negative-NBIs are tangentially injected into LHD plasmas. The positive-ion-based NBIs with low energy of 40 keV were perpendicularly injected for ion heating [27]. In 2010, second perpendicular-NBI

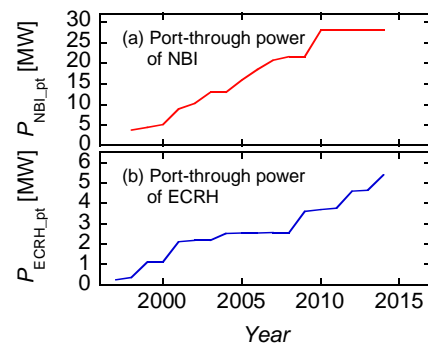


FIG. 1. The history of the port-through power of (a) the NBI and (b) the ECRH in the LHD.

with the beam energy of 40 keV was installed in the LHD and the total-port through power of perpendicular-NBIs reached 12 MW. An ECRH system with eight gyrotrons has been operated for preionization and plasma heating [28]. Of these, the high power 77-GHz gyrotrons with the output power of more than 1 MW have been operated since the experimental campaign in 2007 [28, 29]. In the present state, three 77 GHz gyrotrons are operational for plasma experiments. In addition to these 77 GHz tubes, two high power gyrotrons (154 GHz, 1 MW) were installed in FY2012 and FY2014 for high density plasma heating [13]. The configuration of the ECRH transmission line has also been changed to enhance the central heating capability. In the present condition, four of the five high power ECRH antennas are set at the horizontal ports, then the deeper injection angle of the beams and the smaller beam refraction compared with the injection from the vertical port have been realized. The heating capability on the LHD has been upgraded year by year. Figure 1 shows the history of the port-through power of (a) the NBI and (b) the ECRH in the LHD. The total injection power of the NBI and the ECRH reached 28 MW and 5.4 MW, respectively.

3. Extension of High Temperature Regime

3.1. Response of Temperature Gradients to ECRH

Figure 2 shows (a) the latest high-temperature operational regime in the LHD, and (b) the radial profiles of T_i , T_e , and n_e for a typical high-temperature plasma associated with the e-ITB and the i-ITB. The variable r_{eff}/a_{99} in the horizontal axis of Fig. 2 (b) is the effective minor radius normalized by the averaged minor radius where 99% of the electron stored energy is confined. We finely adjusted the ECRH injection angle and the EC wave polarization taking account of the actual plasma profiles in real time [30]. In Fig. 2 (a), the gray area was obtained using only NBI or ECRH, the blue area was using both ~ 26 -MW NBI and ~ 2 -MW ECRH, and the yellow area was using both ~ 26 -MW NBI and ~ 5 -MW ECRH. The circles and the squares represent the data with and without the adjustment, respectively. The operational regime was successfully extended mainly due to the upgraded ECRH system and the optimization of the ECRH injection. The global parameters of the discharge of Fig. 2 (b) are: the line-averaged electron density measured using FIR $n_{e,\text{fir}}$ was $1.2 \times 10^{19} \text{ m}^{-3}$; the energy confinement time τ_E was 33 ms; and the volume-averaged beta value $\langle \beta \rangle$ was 0.53%, where the kinetic energy was used for the evaluation of τ_E and $\langle \beta \rangle$. Although the T_i profile was flattened in the central region due to the confinement degradation by increase in T_e/T_i , the i-ITB were maintained (actually, dT_i/dr_{eff} at $r_{\text{eff}}/a_{99} = 0.6$ was 2.5 times larger than that of L mode plasma).

3.2. Comparison of the Response of Temperature Gradients to ECRH

The systematic data of the temperature profiles have been obtained for several confinement states in the LHD. Figure 3 shows the comparison of the temperature, electron density, and the temperature gradient profiles among four different confinement states, (a), (b) the L mode plasma, (c), (d) the i-ITB plasma, (e), (f) the e-ITB plasma, and (g), (h) the i- and e-ITB plasma. The magnetic configuration is the

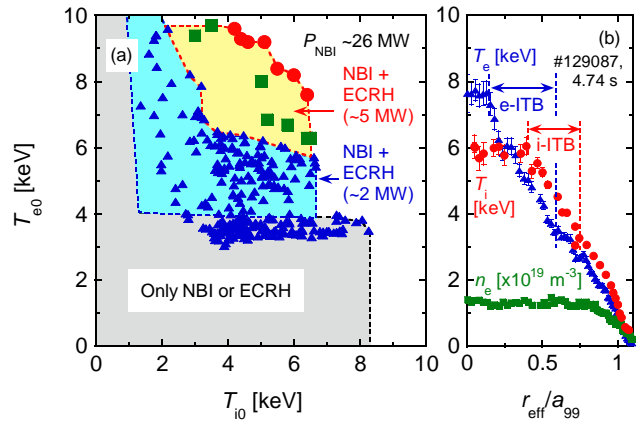


FIG. 2. (a) Extended operational high-temperature regime in the LHD, (b) T_i , T_e , and n_e profiles for a typical high-temperature plasma with the ITBs.

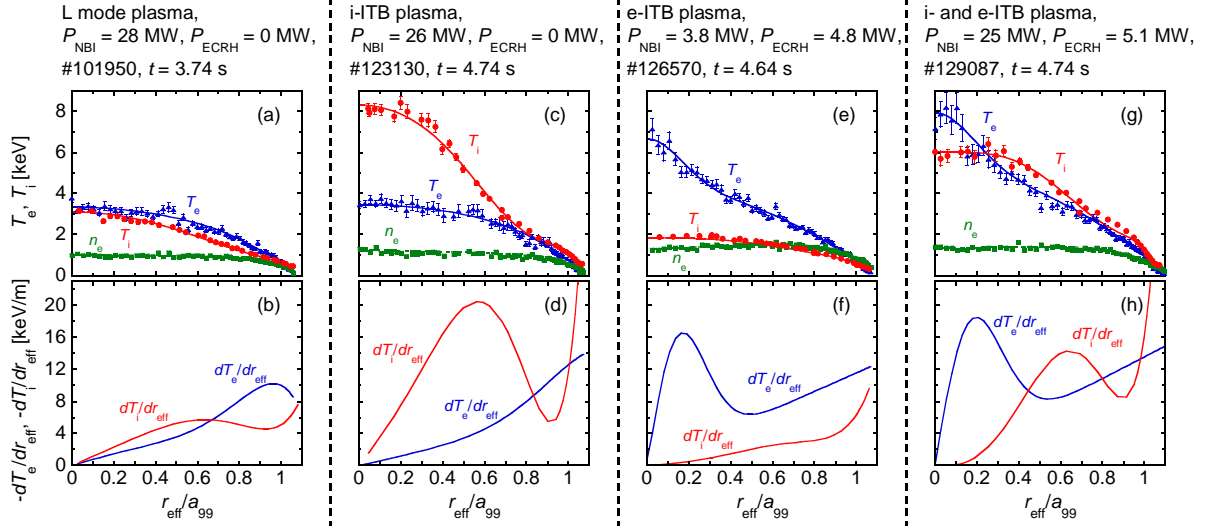


FIG. 3. The comparison of the temperature and the gradient profiles among four different confinement states, (a), (b) the L mode plasma, (c), (d) the i-ITB plasma, (e), (f) the e-ITB plasma, and (g), (h) the i- and e-ITB plasma.

vacuum-magnetic axis R_{ax} of 3.6 m and the toroidal magnetic field on the axis B_t of 2.75-2.85 T. The L mode plasma and the i-ITB plasma were produced using NBI alone (26-28 MW). The e-ITB plasma was mainly heated using the on-axis ECRH (4.8 MW). In addition to the ECRH, a diagnostic NBI (3.8 MW) was injected in order to measure the T_i profile by CXRS. The i- and e-ITB plasma was obtained using both the NBI (25 MW) and the on-axis ECRH (5.1 MW). The n_e profiles were flat except for the e-ITB plasma. The n_e profile of the e-ITB plasma was a slightly hollow shape due to the smaller core fuelling of the NBI compared with the other three cases and the density clamping effect by the on-axis ECRH. The line-averaged density $n_{e, fir}$ was $0.87 \times 10^{19} \text{ m}^{-3}$ for the L mode plasma, $1.0 \times 10^{19} \text{ m}^{-3}$ for the i-ITB plasma, $1.3 \times 10^{19} \text{ m}^{-3}$ for the e-ITB plasma, and $1.2 \times 10^{19} \text{ m}^{-3}$ for the i- and e-ITB plasma. Here we focus on the description of the profile characteristics of the i- and e-ITB plasma. Please see the references for the details of the confinement characteristics for the i-ITB [18-22] and the e-ITB [10-13] in the LHD.

For the i- and e-ITB plasma, the ion-ITB structure was maintained in the plasma core region even the peak value of the dT_i/dr_{eff} at $r_{eff}/a_{99} \sim 0.6$ was slightly degraded compared with i-ITB plasma. The T_i profile became almost flattened in the plasma central region. On the other hand, the dT_i/dr_{eff} greatly increased in the plasma edge. Compared with L mode plasma, the dT_e/dr_{eff} was significantly increased in the core region due to the formation of the e-ITB. The shape of the profiles, and especially the position of the local maximum and the local minimum of the dT_e/dr_{eff} and the dT_i/dr_{eff} are approximately the same as the dT/dr_{eff} of the e-ITB plasma and the dT_i/dr_{eff} of the i-ITB, respectively. The ITB positions were different between the T_i and the T_e , namely the e-ITB was formed at the inner position compared with i-ITB. The difference of the ITB positions has been observed in the LHD [8]. As a result, the T_e/T_i tends to be higher in the central region for i- and e-ITB plasmas.

4. Effect of ECRH on Ion Temperature Profile

4.1. Response of Temperature Gradients to ECRH

Simultaneous high T_i and high T_e have been obtained by the on-axis ECRH superposition on the high- T_i plasmas sustained by the NBI. In such condition, a seesaw-like behavior of the ion

thermal transport between core and edge has been observed. As shown in Fig. 3 (g), the T_i gradient was decreased in the plasma core due to the superposition of the on-axis ECRH. On the other hand, the T_i gradient increased in the plasma edge. Here we discuss the details of the effect of the ECRH on the T_i profile.

Figure 4 shows the time evolution of (a) the port through power of the NBI and the ECRH, (b) the line-averaged-electron density, (c), (d) the gradient of T_e and T_i at $r_{\text{eff}}/a_{99} = 0.31$ and 0.98 , and (e) the kinetic energy for electron and ion. The on-axis ECRH was superposed stepwise up to ~ 5 MW on the NBI-sustained plasma. The ray-tracing calculation showed that more than 90% of the ECRH power was deposited inside $r_{\text{eff}}/a_{99} = 0.2$ at 4.74 s. Both in the plasma core and the edge, T_e and the gradient increased with increase in the ECRH power. On the other hand, the gradient of T_i was degraded in the plasma core with increase in the ECRH power. In contrast, the T_i gradient was improved at the plasma edge. We confirmed that the ion heating power of the NBI increased during the ECRH superposition because of the increase in the beam critical energy by T_e increase. However, the increment of the ion heating power was 11% from the comparison between $t = 4.24$ s and 4.74 s, and this cannot explain the doubled dT_i/dr_{eff} at the edge. This edge gradient increase is considered to be a different phenomenon from the so-called confinement transition because the edge gradient changed linearly with the step-by-step-increased ECRH power. The ion kinetic energy was also increased with the increase in the ECRH power due to the steepened T_i gradient at the edge.

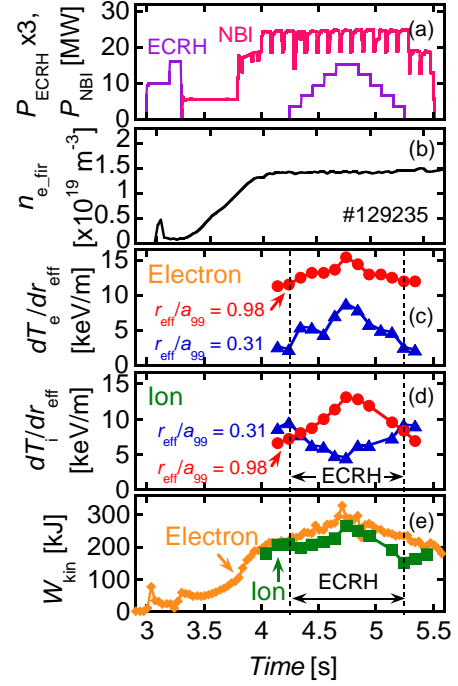


FIG. 4. The time evolution of (a) the port through power of the NBI and the ECRH, (b) the line-averaged-electron density, (c), (d) the gradient of T_e and T_i at $r_{\text{eff}}/a_{99} = 0.31$ and 0.98 , and (e) the kinetic energy for electron and ion.

4.2. Dependence of Change in Temperature Profiles during ECRH on Electron Density

As introduced in Sec. 4.1, temperature profiles are clearly affected by the ECRH not only for the electron but also for the ion. Here the temperature profile modification by the ECRH and the dependence on n_e range are introduced. Figure 5 shows the radial temperature profiles of (a) electron and (b) ion before and during the ECRH (5.1 MW), and (c) the time evolution of the relation between T_e and T_i at several radial positions during the ECRH power changing for lower line-averaged n_e of $1.0 \times 10^{19} \text{ m}^{-3}$, and (d)-(f) for higher line-averaged n_e of $1.4 \times 10^{19} \text{ m}^{-3}$. The dashed lines in Fig. 5 (c), (f) represent $T_e/T_i = 1$. In both discharges, five ECRH beams of 1 MW each were superposed in stepwise on the NBI-sustained plasmas as shown in Fig. 4 (a) and were focused on the axis.

For the lower n_e case, T_e and the gradients increased in the entire plasma region, and especially in the core due to the ITB. The increase in the gradient outside of the ITB was due to the increase in the electron heat flux by the ECRH superposition. On the other hand, a seesaw-like behaviour was observed in the T_i profile. Here T_e/T_i exceeded 1 in the core region by the centre-focused ECRH, then T_i and its gradient degraded, resulting in the largely flattened T_i profile. In contrast, the dT_i/dr increased in the edge, leading to the T_i increase in the wide region except for the plasma centre. For the higher n_e case, the responses of T_e and T_i

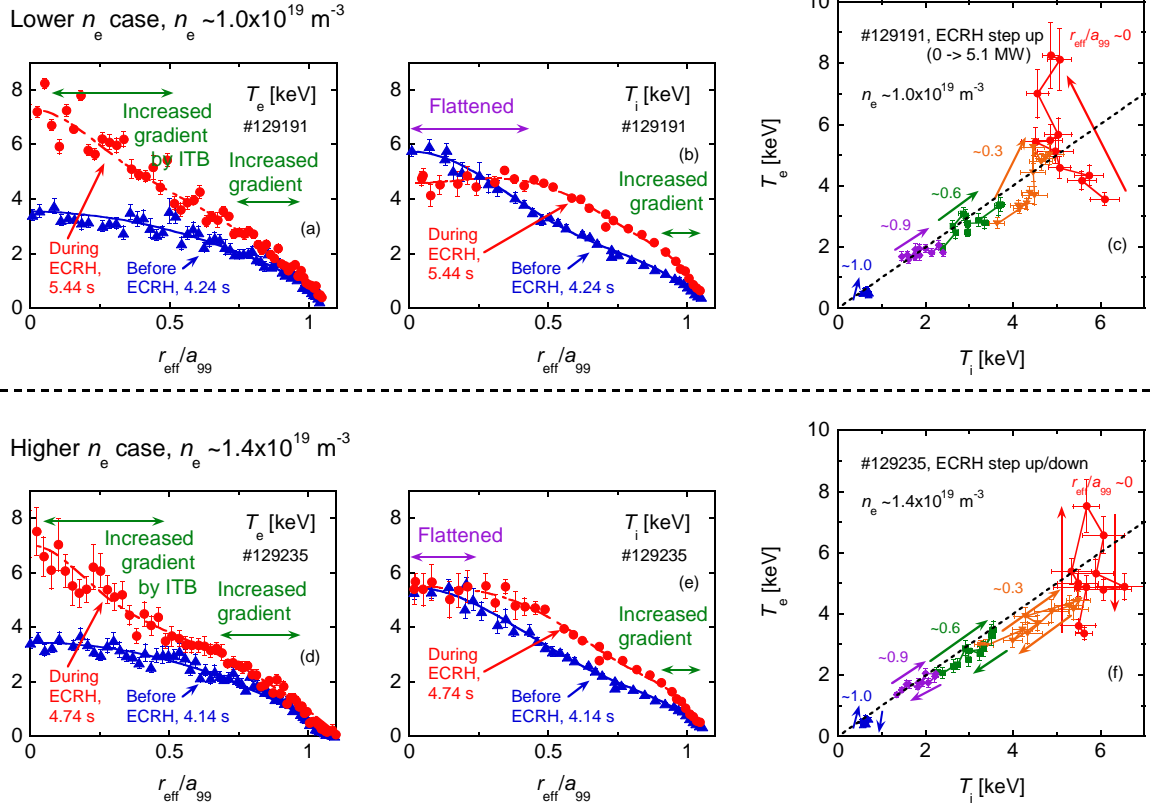


FIG. 5. The radial temperature profiles of (a) electron and (b) ion before and during the ECRH (5.1 MW), and (c) the time evolution of the relation between T_e and T_i at several radial positions during the ECRH power changing for lower line-averaged n_e of $1.0 \times 10^{19} \text{ m}^{-3}$, and (d)-(f) for higher line-averaged n_e of $1.4 \times 10^{19} \text{ m}^{-3}$.

profiles to the ECRH were similar to lower n_e situation but the decrease in T_i was not observed in the plasma centre. In the study, the core T_i was found to be determined by the competition between core-confinement degradation and the edge-confinement improvement. Although the dT_i/dr around the centre degraded, T_i itself was not decreased even though T_e/T_i exceeded 1 due to (1) the improvement of peripheral T_i gradient and (2) the i-ITB was maintained in the more inward side compared with lower n_e by the decrease of the flattening area.

Figure 6 shows the radial profiles of the increment in T_i by the ECRH superposition for two n_e cases.

These are obtained as subtractions of data in Fig. 5 (b) and (e), respectively. The T_i increment in the inner half of the minor radius were different between the two conditions. Core T_i can be increased by the on-axis ECRH in higher n_e plasmas. This is a beneficial feature for a high T_i scenario for high density plasmas under electron heating dominant like DEMO. The reason has not been clarified why the higher n_e was better to sustain higher core T_i in the high T_e/T_i situation. Actually the equipartition heating power was -0.1 MW (minus means the heat flux moved to electron from ion) on the other hand the absorbed-ion-heating power of the NBI was 7.0 MW for $n_e = 1.4 \times 10^{19} \text{ m}^{-3}$ case. Thus the contribution of the equipartition heat flux is

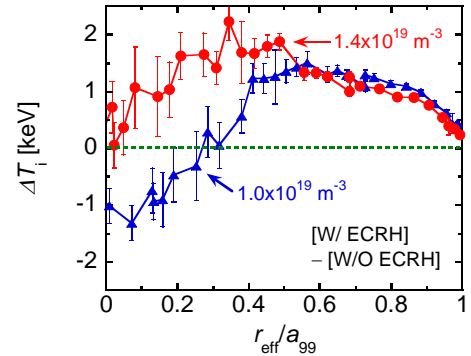


FIG. 6. The radial profiles of the increment in T_i by ECRH superposition for two n_e cases.

considerably small. The systematic experiment with the various n_e conditions is required to explain the results.

5. Discussion

5.1. Change in Ion Thermal Diffusivity by ECRH

In the previous section, the modification of the ion temperature profile by the ECRH was discussed. Here the temporal change in the ion thermal confinement during the ECRH superposition in the present discharge is treated. Dynamic transport analysis taking into account the slowing-down effect of the NBI deposition is suitable when the plasma parameters change transiently in the discharge [31]. Figure 7 shows the flux-gradient relation between Q_i/n_e and dT_i/dr_{eff} at (a) $r_{\text{eff}}/a_{99} = 0.31$ and (b) 0.98, where Q_i is the ion-heat flux. The slope in the relation between Q_i/n_e and dT_i/dr_{eff} correspond to χ_i . The target discharge is that shown in Fig. 4, namely the on-axis ECRH of 1-MW each was superposed in stepwise up to 5.1 MW on the plasma sustained by the 25-MW NBI. Two branches of χ_i are illustrated in each figure. One is the branch at the timing of no ECRH superposition (25-MW NBI alone) and another is that of full ECRH superposition (both 5.1-MW ECRH and 25-MW NBI). As shown in Fig. 7 (a), dT_i/dr_{eff} became half with the increase in the power of the ECRH superposition despite the small change of Q_i/n_e at $r_{\text{eff}}/a_{99} = 0.31$. This indicates that the ion-thermal confinement was degraded. Then the ion thermal confinement returned to the original confinement state with the decrease in the ECRH power. On the other hand, the ion thermal confinement at the edge improved due to the on-axis ECRH and the dependence of the ion thermal confinement on the ECRH power was opposite to that at the core region.

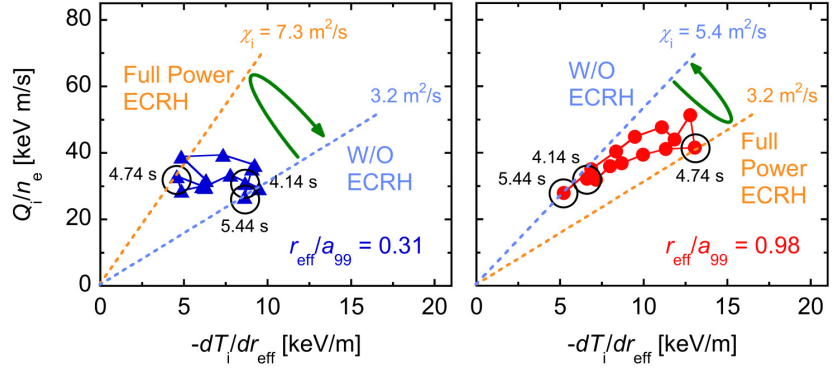


FIG. 7. The flux-gradient relation between Q_i/n_e and dT_i/dr_{eff} at (a) $r_{\text{eff}}/a_{99} = 0.31$ and (b) 0.98.

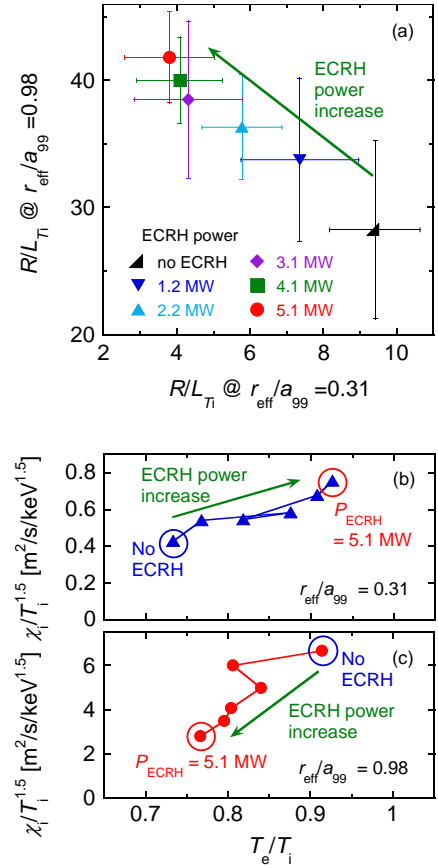


FIG. 8. (a) The relation of the normalized scale length of the ion temperature gradient R/L_{T_i} between the core and the edge, the ion thermal diffusivity normalized by the Gyro-Bohm factor $\chi_i/T_i^{1.5}$ at (b) the core and (c) the edge.

Figure 8 shows (a) the relation of the normalized scale length of the ion temperature gradient R/L_{T_i} between the core and the edge, the ion thermal diffusivity normalized by the Gyro-Bohm factor $\chi_i/T_i^{1.5}$ at (b) the core, and (c) the edge. The ion thermal diffusivity was evaluated by a power balance analysis. R/L_{T_i} degraded by 56% at the core but improved by 50% at the edge. The normalized ion thermal diffusivity was small when the T_e/T_i was small both at the core and the edge even though the dependence on P_{ECRH} was opposite. The

causality of the change in the ion thermal transport is supposed to be different between the core and the edge. In the plasma central region, T_e increased by the ECRH, then T_e/T_i increased and the ion thermal transport was considered to be degraded. On the other hand, T_i increased at the edge by the ECRH, accordingly, T_e/T_i decreased. The normalized ion thermal diffusivity $\chi_i/T_i^{1.5}$ at $r_{\text{eff}}/a_{99} = 0.98$ was reduced by 70%. The improvement of the ion thermal transport at the edge led to the increase in T_i in the entire plasma region and the increase in the ion kinetic energy even though the core ion thermal transport was degraded.

The ambipolar radial electric field E_{r_am} and the neoclassical ion thermal diffusivity χ_{i_NC} were calculated using DCOM code [32]. Although the E_r was not measured for the plasmas presented here, the formation of the negative E_r was confirmed in the i-ITB plasmas [21]. Also the positive E_r formation was found in the plasma with both e- and i-ITBs [8]. Figure 9 shows the dependence of (a) the E_{r_am} and (b) the χ_{i_NC} on the superposed ECRH power at $r_{\text{eff}}/a_{99} = 0.31$ and 0.98. The core E_{r_am} changed from negative to positive with increase in the ECRH power (with increase in the T_e/T_i). The core χ_{i_NC} increased up to $P_{\text{ECRH}} = 3$ MW due to the decrease in the negative E_{r_am} and was significantly decreased in $P_{\text{ECRH}} > 4$ MW due to the presence of the large value of the E_{r_am} in the electron-root. The effect of the ECRH on the edge E_{r_am} and the χ_{i_NC} looks minor compared with the core region. On the other hand, the experimental χ_i from the power balance analysis increased with increase in the T_e/T_i as shown in the Fig. 8 (b) and (c). The tendency of the χ_i on the ECRH power was quite different between the experiment and the neoclassical calculation thus the ion thermal confinement of the plasma presented here is considered to be strongly modified by the anomalous transport depending on the T_e/T_i .

5.2. Response of Fluctuation to ECRH

The effect of the ECRH on the behavior of the fluctuation was also investigated. Figures 10 (a) and (b) are the time and space dependent cross correlation function [33, 34] defined as

$$C(\tau, \rho) = \frac{\langle \tilde{n}_e(t, \rho) \tilde{n}_e(t - \tau, \rho_0) \rangle}{\sqrt{\langle \tilde{n}_e^2(t, \rho) \rangle \langle \tilde{n}_e^2(t, \rho_0) \rangle}}, \quad (1)$$

where \tilde{n}_e denotes the n_e fluctuation having an intermediate frequency range ($f < 20$ kHz). The n_e fluctuation was measured using microwave frequency comb Doppler reflectometer [35]. The reference radius ρ_0 is taken as the outer most measurement point of the reflectometer. The angular bracket indicates the time average. The strength of the spatial correlation can be evaluated as the time average of the envelope of the cross correlation function defined as

$$\frac{1}{N} \sum_{\tau} |C(\tau, \rho)|_E, \quad (2)$$

where $|*|_E$ denotes the envelope of the signal. This quantity is shown in Fig. 10 (c) as a function of the ECRH injection power. These data were obtained for the discharge introduced in Fig. 4. As can be seen from the figure, the radial structure of the edge fluctuation was

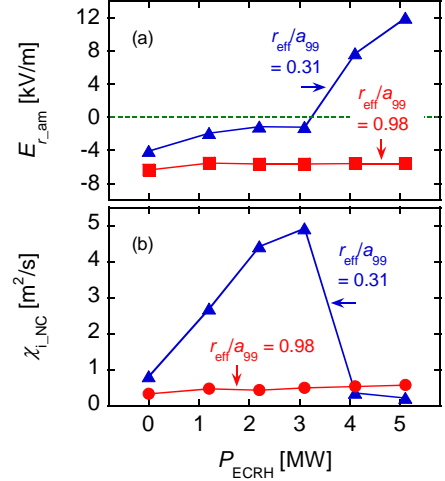


FIG. 9. The dependence of (a) the E_{r_am} and (b) the χ_{i_NC} on the superposed ECRH power at $r_{\text{eff}}/a_{99} = 0.31$ and 0.98.

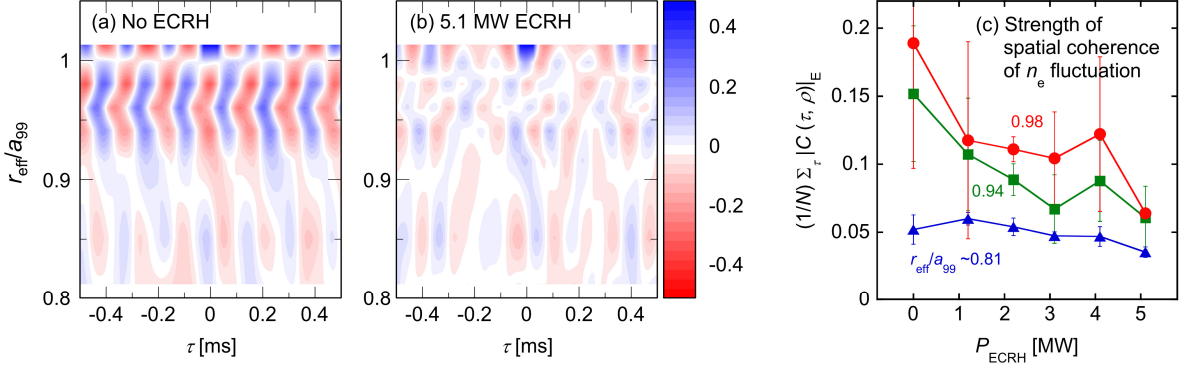


FIG. 10. The time and space dependent cross correlation function for the timing of (a) no ECRH and (b) 5.1 MW ECRH, (c) the ECRH power dependence of the time average of the envelope of the cross correlation function.

segmentalized by the on-axis ECRH superposition. This indicates that the correlation length of the fluctuation decreased. Also, the strength of the spatial correlation decreased with increase in the ECRH power due to the change in the correlation length and the correlation time.

We also measured the n_e fluctuation using two dimensional phase contrast imaging (2-D PCI) [36]. Figure 11 shows the radial profiles of (a), (b) the phase velocity, (c), (d) the amplitude of the n_e fluctuation, and (e) $\chi_i/T_i^{1.5}$ before and during the ECRH (5.1 MW). The data in Fig. 11 (a) and (c) was obtained using 2-D PCI at the lower side of the equatorial plane and (b) and (d) were at the upper side. The ion thermal diffusivity was obtained by the power balance analysis. The up-down asymmetry of the fluctuation structure was observed in the experiments. This is considered due to the tilting of the turbulence eddy [36, 37]. The phase velocity in the ion-diamagnetic direction increased in the entire plasma region by the ECRH superposition. This indicates that the positive E_r increased. The fluctuation amplitude was increased in the inner half of the plasma radius during the ECRH superposition. Especially, the response of the fluctuation amplitude in the upper side to the ECRH coincidentally correlated with the change in $\chi_i/T_i^{1.5}$, namely, the amplitude and the thermal diffusivity were increased in the inner half of the plasma radius and were decreased in the outer half of the radius.

6. Summary

Simultaneous high T_i and high T_e regime was successfully extended in the LHD. The integration of high T_i and high T_e was realized due to the establishment of the i-ITB and e-ITB simultaneously by the NBI and the ECRH mix. It was found that the ion thermal confinement

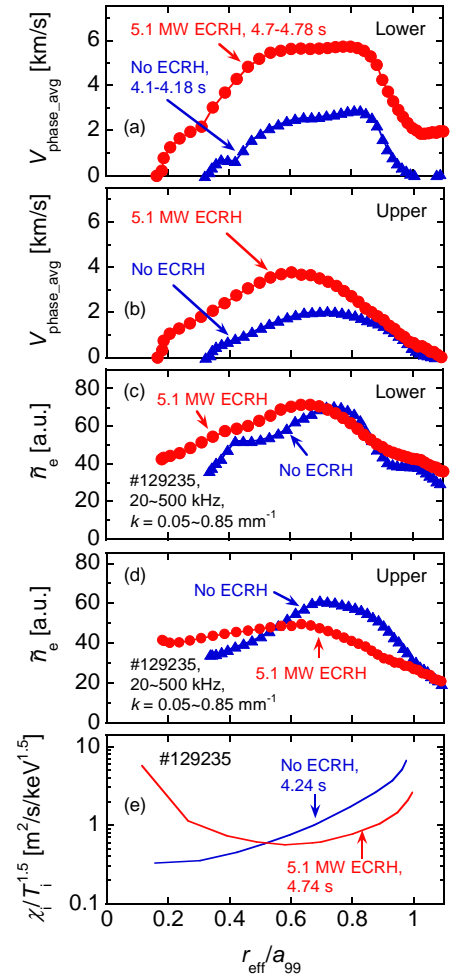


FIG. 11. The radial profiles of (a), (b) the phase velocity, (c), (d) the amplitude of the n_e fluctuation, and (e) $\chi_i/T_i^{1.5}$ before and during the ECRH (5.1 MW). (a) and (c) were obtained at the lower side of the equatorial plane and (b) and (d) were at the upper side.

was improved at the plasma edge by the on-axis ECRH superposition on the NBI-sustained plasmas although the ion thermal confinement was degraded in the plasma core with increase of T_e/T_i . The normalized ion thermal diffusivity $\chi_i/T_i^{1.5}$ at the plasma edge was reduced by 70%. The improvement of the ion thermal confinement at the edge led to increase in T_i in the entire plasma region, especially for higher n_e plasmas, even though the core transport was degraded. The feature is beneficial for a high T_i scenario for high density plasmas under electron heating dominant such as DEMO. The high temperature plasmas obtained in the work was of still low density compared with the reactor ($\sim 10^{20} \text{ m}^{-3}$). The confinement characteristics of ITB plasmas with higher density will be investigated in the LHD. Also scenarios of an impurity control have to be established toward DEMO. High temperature plasmas with a low collisionality as presented in the paper are beneficial for the investigation of the impurity control [38].

The structure of the n_e fluctuation at the edge was segmentalized and the amplitude at the upper side of the equatorial plane decreased in the outer half of the plasma. It has not been clarified why the edge ion thermal confinement improved during the on-axis ECRH. The accumulation of the systematic data is necessary. The present data were obtained for the discharges only with the on-axis ECRH superposition. We need to conduct experiments using an off-axis ECRH to clarify the ECRH effect on the edge-core interaction of the ion thermal confinement.

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